

# Lecture 10: The Physics of Neutron Moderation

CBE 30235: Introduction to Nuclear Engineering — D. T. Leighton

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## 1 Introduction: The Fork in the Road (Fast vs. Thermal)

In Lecture 9, we defined the cross section ( $\sigma$ ). In this lecture we shall show that for Uranium-235, the probability of fission is massive at low energies ( $\sim 580$  barns at 0.025 eV) but tiny at high energies ( $\sim 1$  barn at 2 MeV).

When a fission event occurs, the new neutrons are born **Fast** ( $E \approx 2$  MeV). This presents the nuclear engineer with two fundamental choices:

### 1.1 Option A: The Thermal Reactor (The focus of this course)

We deliberately slow the neutrons down using a **Moderator** (water, graphite) to take advantage of the massive low-energy cross section.

- **Pros:** We can use natural or slightly enriched uranium (3-5%). The physics is easier to control.
- **Cons:** We lose neutrons to absorption during the slowing-down process.
- **Examples:** LWR (USA), CANDU (Canada), HTGR.

### 1.2 Option B: The Fast Reactor

We keep the neutrons moving at high speeds (no moderator) to fission  $^{238}\text{U}$  or breed Plutonium.

- **Pros:** At high energies, the neutron yield per fission ( $\nu$ ) is higher, allowing for "Breeding" (creating more fuel than is consumed). This allows us to close the fuel cycle (as done in France/Russia).
- **Cons:** Since the cross section is so small ( $\sim 1$  barn), the fuel must be highly enriched ( $> 20\%$ ) to sustain a chain reaction. The coolant cannot contain hydrogen (usually liquid Sodium or Lead).

**Decision:** For the next several weeks, we will focus exclusively on **Option A**. To understand Thermal Reactors, we must understand the mechanics of how a neutron loses energy. This process is called **Moderation**.

## 2 The Life Cycle of a Neutron (Part 1)

In Lecture 9, we defined the probability of interaction (the cross section,  $\sigma$ ). Today, we look at the consequences of those interactions. Our goal in a thermal nuclear reactor is to take a "fast" neutron (born at high energy from fission) and slow it down to "thermal" energies where it can easily cause more fission.

This involves distinct physics problems:

1. **Moderation:** How does a neutron lose energy when it hits a nucleus? (Scattering mechanics).
2. **Probability of Scattering:** How likely is a collision at different energies?
3. **Absorption (Losses):** How do we avoid the "traps" (resonances) while slowing down?

## 3 Dynamics of Neutron Scattering (Moderation)

*Primary Reference: Lamarsh, 4th Ed., Section 3.5*

To achieve the "Thermal Reactor" described in Option A, we must reduce the neutron energy from 2 MeV to 0.025 eV. This is a factor of  $10^8$  reduction.

### 3.1 Energy Loss in a Single Elastic Collision

Consider a neutron (mass 1) hitting a target nucleus (mass  $A$ ) in an elastic "billiard ball" collision. Conservation of momentum and energy dictates how much energy is transferred.

#### Visualizing the Physics: The "Head-On" Collision ( $A = 1$ )

The mechanics of moderation are best understood by looking at the specific case of a neutron hitting a Hydrogen nucleus ( $A = 1$ ) head-on. The easiest way to solve this is to shift our perspective to the **Center of Mass (COM)** frame of reference.

1. **Lab Frame (Before):** The neutron moves at velocity  $v$ . The proton is stationary. The "average" velocity of the system (Center of Mass) is  $v_{cm} = v/2$ .
2. **COM Frame:** We move along with the center of mass.
  - The neutron appears to move right at  $v - v/2 = +v/2$ .
  - The proton appears to move left at  $0 - v/2 = -v/2$ .
3. **The Collision:** Since momentum must be zero in the COM frame and energy is conserved, the particles simply bounce off with the same speed but opposite direction.
  - Neutron rebound velocity (COM) =  $-v/2$ .
4. **Lab Frame (After):** We shift back to the lab frame by adding  $v_{cm}$  to the result.
  - Neutron Final Velocity =  $(-v/2)_{rebound} + (+v/2)_{frame} = \mathbf{0}$ .

**Conclusion:** In a head-on collision with Hydrogen, the neutron transfers **100%** of its momentum and energy to the proton, stopping dead in its tracks.

## Generalizing to Mass $A$

For heavier nuclei ( $A > 1$ ), the center of mass moves much slower than  $v/2$ , so the neutron can never lose all its energy. The maximum possible energy loss is determined by the **Collision Parameter**,  $\alpha$ :

$$\alpha = \left( \frac{A-1}{A+1} \right)^2 \quad (1)$$

The neutron's energy after one collision will fall somewhere in the range:

$$\alpha E_{in} \leq E' \leq E_{in} \quad (2)$$

*Note:*

- **Hydrogen** ( $A = 1$ ):  $\alpha = 0$ . The neutron can lose up to 100% of its energy (as derived above).
- **Lead** ( $A = 208$ ):  $\alpha \approx 0.98$ . The neutron retains at least 98% of its energy even in a perfect head-on collision.

## 3.2 Average Logarithmic Energy Decrement ( $\xi$ )

Since neutrons scatter at random angles, we care about the *average* behavior. We define  $\xi$  (xi) as the average loss in the natural log of energy per collision:

$$\xi = \ln \left( \frac{E_{in}}{E_{out}} \right)_{\text{average}} \approx \frac{2}{A+2/3} \quad (\text{for } A > 1) \quad (3)$$

This parameter dictates our choice of moderator:

- **Water (H):**  $\xi = 1.0$ . Requires  $\sim 18$  collisions to thermalize.
- **Graphite (C):**  $\xi = 0.158$ . Requires  $\sim 115$  collisions.
- **Uranium (U):**  $\xi = 0.008$ . Requires  $\sim 2170$  collisions.

## 3.3 Energy Dependence of Elastic Scattering

So far, we have discussed *how much* energy is lost if a collision occurs. But how *likely* is the collision?

For most good moderators (like Hydrogen, Carbon, and Oxygen) in the energy range of interest (1 eV to 100 keV), the elastic scattering cross-section ( $\sigma_s$ ) is relatively constant.

- This is often called **Potential Scattering**.
- The neutron essentially sees the nucleus as a hard sphere with cross-section  $\sigma_s \approx 4\pi R^2$ , where  $R$  is the nuclear radius.
- **Significance:** Unlike absorption (which varies wildly, see Section 4), the scattering probability is reliable. This creates a steady "staircase" of energy loss, allowing the neutron to reliably march down from MeV energies toward thermal energies.

### 3.4 Inelastic Scattering: The "Heavy Lifter" at High Energy

You might ask: "Doesn't inelastic scattering lose even more energy?" Yes. In an inelastic collision ( $n + A \rightarrow n' + A^*$ ), the neutron gives up a massive amount of kinetic energy to excite the nucleus.

- **Significance:** This mechanism is very effective for **Heavy Nuclei** (like Uranium or Iron) at high energies ( $> 1$  MeV).
- **The Limitation:** It is a **Threshold Reaction**. It can only occur if the neutron has enough energy to reach the first excited state of the nucleus.
  - For U-238, the threshold is low ( $\sim 45$  keV).
  - For Oxygen-16 (moderator), the threshold is huge ( $\sim 6$  MeV).

**Conclusion:** Since fission neutrons are born at  $\sim 2$  MeV, they *cannot* inelastically scatter against the Oxygen in the water. We must rely on **Elastic Scattering** to take the neutron the rest of the way down.

## 4 Neutron Absorption (Losses)

*Primary Reference: Lamarsh Section 3.6*

Now that we know *how* the neutron slows down (Section 3), we must examine the treacherous path it must travel. While we want the neutron to scatter, we desperately want to avoid it being **Captured** (Absorbed) before it reaches thermal energy.

### 4.1 The "Valley of Death" (Resonance Region)

Between 1 eV and 10 keV, the capture cross-section of  $^{238}\text{U}$  exhibits a "forest" of sharp, jagged peaks.

- **The Physics:** These peaks correspond to discrete quantum energy levels of the excited compound nucleus ( $^{239}\text{U}^*$ ).
- **The Trap:** In  $^{238}\text{U}$ , these are almost exclusively **Capture Resonances** ( $n, \gamma$ ). If a neutron slows down and lands in one of these energy "traps" (e.g., the massive resonance at 6.67 eV), it is absorbed. It dies without causing fission.
- **The Name:** We call this the "Valley of Death" because neutrons must successfully "jump" over these peaks via elastic scattering to reach the safe thermal region.

### 4.2 Visualizing the Resonances (Interactive Tool)

Since we cannot see this behavior in a static textbook plot, we will use an online nuclear data plotter. *Note: The standard NNDC site is often offline.*

**Primary Tool: KAERI Table of Nuclides** <https://atom.kaeri.re.kr/nuchart/>

**Exercise for Students:**

1. Go to the URL above.
2. In the "Nuclide" search box, type: U238 (and hit Enter).

3. **Choose the Library:** Locate the column labeled **ENDF/B-VIII.0** (this is the US standard) and click on the small + sign to expand the options.
4. **Select the Data:** In the expanded list, locate the row labeled **Capture cross sections** and click the button labeled **Plot**.
5. **Verify Scale:** The graph that appears is the  $(n, \gamma)$  radiative capture cross section. The axes should default to **Log-Log** scale. (If they appear linear, use the settings to switch them to Log).
6. **Observation:** Look at the energy range between  $10^0$  eV and  $10^4$  eV. You will see the massive peaks rising 1000x above the baseline. These are the "Resonance Traps" our moderator design must defeat.

### 4.3 Region 3: The $1/v$ Region (Thermal)

Once the neutron passes the last resonance (at roughly 6 eV), it enters the "safe" thermal region. Here, the absorption cross-section rises smoothly according to the  **$1/v$  Law**:

$$\sigma_a \propto \frac{1}{v} \propto \frac{1}{\sqrt{E}} \quad (4)$$

This massive rise at low energy is what makes thermal reactors possible.

## 5 Summary: The Goal of Lecture 10

We have successfully described the journey of the neutron from birth (high energy) to thermal maturity (low energy).

- We defined the **Moderator** (H, C) and why we use it ( $\xi$ ).
- We analyzed the **Mechanics** of energy loss ( $\alpha$ , COM frame).
- We identified the **Enemy**: Resonance Capture in U-238.

However, we have not yet discussed the most important event: **Death and Rebirth**. What happens when the thermal neutron finally hits a  $^{235}\text{U}$  nucleus?

In **Lecture 11 (Friday)**, we will cover the **Physics of Fission**, the Liquid Drop Model, and the origin of the massive energy release that powers the reactor.